Office for Nuclear Regulation

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WESTINGHOUSE AP1000[®] GENERIC DESIGN ASSESSMENT GDA ISSUE OVATION PLATFORM ADEQUACY OF SAFETY CASE

GI-AP1000-CI-06 REVISION 0

Technical Area		CONTROL AND INSTRUMENTATION			
Related Technical Areas		None			
GDA Issue Reference	GI-AP1000-CI-06		GDA Issue Action Reference	GI-AP1000-CI-06.A1	
GDA Issue	ONR is seeking an adequate safety case for the Ovation platform that supports the Class 2 closed loop controls and the Class 3 manual controls and displays of AP1000. Westinghouse submitted information on the platform but progress on its assessment was delayed due to priority being given to topics relating to the protection systems including the PMS/CIM safety case, CIM/DAS diversity and PMS blocker. A basis of safety case for the Ovation platform and access to it supporting evidence is required. For further guidance, see T15.TO1.01 in Annex 5 of ONR C&I Assessment Report No. 11/006 (draft).				
GDA Issue Action	Westinghouse to facilitate ONR access in the UK to the detailed evidence used to support the basis of safety case for the Ovation platform. With agreement from the Regulator this action may be completed by alternative means.				

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GI-AP1000-CI-06 REVISION 0

Technical Area Related Technical Areas		CONTROL AND INSTRUMENTATION None				
						GDA Issue Reference
GDA Issue Action	Westinghouse to provide a basis of safety case (BSC) that includes a justification of the suitability of the Ovation platform for Class 2 and 3 systems.					

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OVATION PLATFORM ADEQUACY OF SAFETY CASE

GI-AP1000-CI-06 REVISION 0

Technical Area		CONTROL AND INSTRUMENTATION					
Related Technical Areas			None				
GDA Issue Reference	GI-AP1000-CI-	06	GDA Issue Action Reference	GI-AP1000-CI-06.A2			
	The BSC should identify the pedigree of any COTS and pre-developed components and provide a demonstration of the adequacy of the development arrangements. For older components the safety argument might involve use of proven in use arguments and testing rather than a production excellence argument. In either case any compensatory measures undertaken to address shortfalls should be identified in the safety demonstration.						
	The BSC should demonstrate how the design and implementation of the equipment using complex / programmable, components, e.g. microprocessors, ASICs, and Field Programmable Gate Arrays complies with relevant WEC safety principles and standards. Given the programmable nature of such complex devices, SAP ESS.27 a special case procedure for the demonstration of safety that involves the presentation of an argument of production excellence and implementation of independent confidence building measures. Where complex hardware is involved, the BSC should identify how the safety demonstration conforms to ESS.21 and the need for measures such as independent third party assessment.						
	The BSC should include a plan that shows the forward activities, and production of related safety case documentation and evidence. Interim BSCs should be provided, particularly for large complex systems. A BSC for the completed design ¹ should be submitted as soon as reasonably practicable before permission to commence nuclear site construction is sought. A BSC for installation and commissioning would be expected before equipment is delivered to site.						
	Notes 1. Completed design – The design is complete at the point where the:						
	 requirements, specifications, and implementation details (e.g. software coding and circuit diagrams etc.) have been completed; 						
	 production verification and validation activities (i.e. prior to delivery to site) have been completed; 						
	 prototype equipment has been produced and subject to performance and qualification testing; 						
	With agreement from the Regulator this action may be completed by alternative means.						

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